Place: National Science Foundation, 4201 Wilson Blvd., Arlington, VA, Room 380.

Type of Meeting: Closed.

Contact Person: Betty Jones & Costello Brown, National Science Foundation, 4201 Wilson Blvd., Arlington, VA 22230. Telephone: (703) 306–1633.

Purpose of Meeting: To provide advice and recommendations concerning proposals submitted to NSF for financial support.

Agenda: To review and evaluate programs for Comprehensive Partnerships for Minority Student Achievement (CPMSA) as part of the selection process for awards.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information; financial data, such as salaries; and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b(c), (4) and (6) of the Government in the Sunshine Act.

Dated: February 21, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95–4664 Filed 2–24–95; 8:45 am]

BILLING CODE 7555–01–M

Special Emphasis Panel in Materials Research; Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92–463 as amended), the National Science Foundation announces the following meeting:

Name: Special Emphasis Panel in Materials Research.

Date and Time: April 25th and 26th, 1995 @ 8:15 am.

Place: National Science Foundation, Rooms 680, 1020, 1150, 1005, 365, 379, 4201 Wilson Boulevard, Arlington, VA 22230.

Type of Meeting: Closed.

Contact Person: G. X. Tessema, and H. Hollis Wickman, DMR, PDs, Room: 1065, Phone: 703–306–1995.

Purpose of Meeting: To provide advice and recommendations concerning support for research proposals.

Agenda: To review and evaluate CMP proposals.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information, financial data such as salaries, and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b.(c) (4) and (6) of the Government in the Sunshine Act.

Dated: February 21, 1995.

M. Rebecca Winkler,

Committee Management Office.

[FR Doc. 95–4659 Filed 2–24–95; 8:45 am]

BILLING CODE 7555–01–M

Special Emphasis Panel in Materials Research; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92–463, as amended), the National Science Foundation announces the following meeting.

Name: Special Emphasis Panel in Materials Research (#1203).

Date and Time: March 13–15, 1995; 8:00 a.m. to 5:00 p.m.

Place: Florida State University, Tallahassee, FL.

Type of Meeting: Closed.

Contact Person: Dr. Adriaan M. de Graaf, Executive Officer, Division of Materials Research, Room 1065, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230. Telephone: (703) 306–182; FAX (703) 306–0515.

Purpose of Meeting: To provide advice and recommendations concerning the continued support for the National High Magnetic Field Laboratory (NHMFL) being established by Florida State University, the University of Florida, and Los Alamos National Laboratory.

Agenda: To review and evaluate the progress report and proposal for continued funding from the NHMFL.

Reason for Closing: The progress report being reviewed includes information of a proprietary or confidential nature, including technical information; financial data, such as salaries; and personal information concerning individuals associated with the proposal. These matters are exempt under 5 U.S.C. 552b(c), (4) and (6) of the Government in the Sunshine Act.

Dated: February 21, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95–4669 Filed 2–24–95; 8:45 am]

BILLING CODE 7555–01–M

Special Emphasis Panel in Undergraduate Education; Notice of Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92–463, as amended), the National Science Foundation announces the following meeting:

Name: Special Emphasis Panel in Undergraduate Education.

Date and Time: March 16, 1995 7:30 p.m. to 9:00 p.m.; March 17, 1995; 8:30 a.m. to 5:00 p.m.; March 18, 1995; 8:30 a.m. to 5:00 p.m.

Place: Doubletree National Airport Hotel, 300 Army/Navy Drive, Arlington, VA 22202.

Type of Meeting: Closed.

Contact Person: Dr. Jim Lightbourne, Section Head, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230, Telephone: (703) 306–1667.

Purpose of Meeting: To provide advice and recommendations concerning proposals submitted to NSF for financial support.

Agenda: To review and evaluate unsolicited proposals submitted to the Course and Curriculum Development (CCD) Panel Meeting.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information, financial data, such as salaries; and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b.(c) (4) and (6) of the Government in the Sunshine Act.

Dated: February 21, 1995.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 95–4663 Filed 2–24–95; 8:45 am]

BILLING CODE 7555–01–M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–255]

Consumers Power Co. Palisades Plant; Environmental Assessment and Finding of no Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of its regulations to Facility Operating License No. DPR–20, issued to the Consumers Power Company, the licensee, for operation of the Palisades Nuclear Plant. The plant is located at the licensee's site in Van Buren County, Michigan.

Environmental Assessment

Identification of Proposed Action

The proposed action requests an exemption from certain requirements of 10 CFR 50.60, ''Acceptance Criteria for Fracture Prevention Measures for Light-Water Nuclear Power Reactors for Normal Operation,'' to allow application of an alternate methodology to determine the low temperature over-pressure protection (LTOP) setpoint for the Palisades Plant. The proposed alternate methodology is consistent with guidelines developed by the American Society of Mechanical Engineers (ASME) Working Group on Operating Plant Criteria (WGOPC) to define pressure limits during LTOP events that avoid certain unnecessary operational restrictions, provide adequate margins against failure of the reactor pressure vessel, and reduce the potential for unnecessary activation of pressure-relieving devices used for LTOP. These guidelines have been incorporated into Code Case N–514, ''Low Temperature Overpressure Protection,'' which has been approved by the ASME Code Committee. The content of this code case has been incorporated into appendix G of Section XI of the ASME

Code and published in the 1993 Addenda to Section XI.

The philosophy used to develop Code Case N–514 guidelines is to ensure that the LTOP limits are still below the pressure/temperature (P/T) limits for normal operation, but allows the pressure that may occur with activation of pressure-relieving devices to exceed the P/T limits, provided acceptable margins are maintained during these events. This philosophy protects the pressure vessel from LTOP events and still maintains the Technical Specification P/T limits applicable for normal heatup and cooldown in accordance with Appendix G to 10 CFR Part 50 and Sections III and XI of the ASME Code.

The proposed action is in accordance with the licensee's request for exemption dated February 10, 1995.

The Need for the Proposed Action

10 CFR 50.60 states that all light-water nuclear power reactors must meet the fracture toughness and material surveillance program requirements for the reactor coolant pressure boundary as set forth in Appendices G and H to 10 CFR Part 50. Appendix G to 10 CFR Part 50 defines P/T limits during any condition of normal operation, including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected over its service lifetime. 10 CFR 50.60(b) specifies that alternatives to the described requirements in Appendices G and H to 10 CFR Part 50 may be used when an exemption is granted by the Commission under 10 CFR 50.12.

To prevent transients that would produce pressure excursions exceeding the Appendix G P/T limits while the reactor is operating at low temperatures, the licensee installed an LTOP system. The LTOP system includes pressure-relieving devices in the form of power-operated relief valves (PORVs) that are set at a pressure low enough that if a transient occurred while the coolant temperature is below the LTOP enabling temperature, they would prevent the pressure in the reactor vessel from exceeding the Appendix G P/T limits. To prevent these valves from lifting as a result of normal operating pressure surges (e.g., reactor coolant pump starting, and shifting operating charging pumps) with the reactor coolant system in a water solid condition, the operating pressure must be maintained below the PORV setpoint.

In addition, in order to prevent cavitation of a reactor coolant pump, the operator must maintain a differential pressure across the reactor coolant pump seals. Hence, the licensee must operate the plant in a pressure window that is defined as the difference between the minimum required pressure to start a reactor coolant pump and the operating margin to prevent lifting of the PORVs due to normal operating pressure surges. The licensee LTOP analysis indicates that using the Appendix G safety margins to determine the PORV setpoint would result in a pressure setpoing within its operating window, but there would be no margin for normal operating pressure surges. Therefore, operating with these limits could result in the lifting of the PORVs and cavitation of the reactor coolant pumps during normal operation. Therefore, the licensee proposed that in determining the PORV setpoint for LTOP events for Palisades, the allowable pressure be determined using the safety margins developed in an alternate methodology in lieu of the safety margins required by Appendix G to 10 CFR Part 50. The alternate methodology is consistent with ASME Code Case N–514. The content of this code case has been incorporated into Appendix G of Section XI of the ASME Code and published in the 1993 Addenda to Section XI.

An exemption from 10 CFR 50.60 is required to use the alternate methodology for calculating the maximum allowable pressure for LTOP considerations.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the licensee's application. Appendix G of the ASME Code requires that the P/T limits be calculated: (a) using a safety factor of 2 on the principal membrane (pressure) stresses, (b) assuming a flaw at the surface with a depth of one-quarter of the vessel wall thickness and a length of 6 times its depth, and (c) using a conservative fracture toughness curve that is based on the lower bound of static, dynamic, and crack arrest fracture toughness tests on material similar to the Palisades reactor vessel material.

In determining the PORV setpoint for LTOP events, the licensee proposed to use safety margins based on an alternate methodology consistent with the proposed ASME Code Case N–514 guidelines. The ASME Code Case N–514 allows determination of the setpoint for LTOP events such that the maximum pressure in the vessel would not exceed 110% of the P/T limits of the existing ASME Appendix G. This results in a safety factor of 1.8 on the principal membrane stresses. All other factors, including assumed flaw size and fracture toughness, remain the same. Although this methodology would reduce the safety factor on the principal membrane stresses, use of the proposed criteria will provide adequate margins of safety to the reactor vessel during LTOP transients.

Because adequate safety margins will be maintained, the change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that this proposed action would result in no significant radiological environmental impact.

With regard to potential nonradiological impacts, the proposed action involves use of more realistic safety margins for determining the PORV setpoint during LTOP events. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed exemption.

Alternative to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. The principal alternative would be to deny the proposed action. Denial of the exemption would not reduce environmental impacts associated with the facility.

Alternative Use of Resources

This action did not involve the use of any resources not previously considered in the Final Environmental Statement related to operation of the Palisades Plant, dated June 1972, and its addendum dated February 1978.

Agencies and Persons Consulted

In accordance with its stated policy, the staff consulted with the Michigan State official regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the foregoing environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to

prepare an environmental impact statement for the proposed action.

For further details with respect to this action, see the request for exemption dated February 10, 1995, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC and at the local public document room located at the Van Wylen Library, Hope College, Holland, MI 49423.

Dated at Rockville, Maryland, this 21st day of February, 1995.

For the Nuclear Regulatory Commission.

John N. Hannon,

Director, Project Directorate III–1, Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.

[FR Doc. 95–4730 Filed 2–24–95; 8:45 am]

BILLING CODE 7590–01–M

[Docket Nos. 50–272 and 50–311]

Public Service Electric and Gas Co.; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. DPR–70 and DPR–75, issued to Public Service Electric and Gas Company, et al. (PSE&G or the licensee) for operation of Salem Nuclear Generating Station (SNGS), Units 1 and 2, located in Salem County, New Jersey.

Environmental Assessment

Identification of the Proposed Action

By letter dated April 16, 1993 (NLR–N83042), PSE&G requested a license amendment to reflect changes to the Updated Final Safety Analysis Report (USFAR) for Salem, Units 1 and 2. The proposed UFSAR change would add an exception to a general statement in the containment isolation system description. The general statement is that automatic containment isolation valves that receive signals to close, fail closed on loss of air or power. The proposed exception would apply to the outboard isolation valves for the control air system. These four valves (11, 12, 21, and 22CA330, collectively identified as CA–330) fail closed on loss of air but fail as-is upon loss of the vital 125 VDC power supply to their solenoid control valves.

Need for Proposed Action

The revision of the licensing basis is needed to exempt the CA–330 valves from the general statement in the USFAR that automatic containment isolation valves that receive signals to close, fail closed on loss of air or power. The staff has examined the design of the isolation system for the control air header piping penetration. With the exception of the failure position for valve CA–330 on loss of its 125 VDC power supply, the design meets all applicable criteria. Failure of the 125 VDC power supply results in a slight degradation in containment isolation reliability. Upon failure of the 125 VDC power supply, the valve will remain in the ''as is'' position. Since the valve is normally open, this means that the valve will stay open and will not close on an isolation signal or loss of air header pressure. For all other single failures, the valve will automatically close upon either loss of air or an isolation signal.

Environmental Impacts of the Proposed Action

The staff has evaluated the conditions for the ''as-is'' failure and finds that the reduction in safety margin due to this condition is acceptably small. First of all, there is a check valve in series with the air-operated valve so that containment integrity is maintained at all times. Secondly, the probability of loss of air pressure is quite low since the normal air supply is backed up with a safety grade supply which is activated automatically upon sensing low air pressure. With an operable air supply, the penetration is not a containment leak path since the air pressure is greater than the peak calculated containment pressure.

The change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does involve features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternate Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Salem Nuclear Generating Station, Units 1 and 2, dated April 1973.

Agencies and Person Contacted

In accordance with its stated policy, the staff consulted with the New Jersey State official regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the application for license amendments dated April 16, 1993, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and the local public document room located at the Salem Free Public Library, 112 West Broadway, Salem, NJ 08079.

Dated at Rockville, Maryland, this 21st day of February 1995.

For the Nuclear Regulatory Commission.

John F. Stolz,

Director, Project Directorate I–2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 95–4731 Filed 2–24–95; 8:45 am]

BILLING CODE 7590–01–M

SECURITIES AND EXCHANGE COMMISSION

[Release No. 35–26233]

Filings Under the Public Utility Holding Company Act of 1935, As Amended (''Act'')

February 17, 1995.

Notice is hereby given that the following filing(s) has/have been made with the Commission pursuant to