NUCLEAR REGULATORY COMMISSION

Nuclear Safety Research Review Committee

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of change of meeting schedule.

As previously announced, the Nuclear Safety Research Review Committee (NSRRC) will hold its next meeting on November 14–15, 1996. The purpose of the present notice is to provide a revised schedule, reflecting a change in the meeting time for the second day of the meeting. The meeting will now take place from 1:00 p.m. to 5:00 p.m. on the 14th and from 7:30 a.m. to 10:45 a.m. on the 15th. The location of the meeting will still be in Room T–10A1, Two White Flint North (TWFN) Building, 11545 Rockville Pike, Rockville, MD.

The meeting will be held in accordance with the requirements of the Federal Advisory Committee Act (FACA) and will be open to public attendance. The NSRRC provides advice to the Director of the Office of Nuclear Regulatory Research (RES) on matters of overall management importance in the direction of the NRC's program of nuclear safety research. The main purpose of this meeting will be: (1) to evaluate the value and contributions of the NSRRC in carrying out the NRC's mission and to develop a set of criteria under which the performance of the NSRRC could be evaluated in the future; (2) to discuss the roles of the NSRRC and the Advisory Committee for Reactor Safeguards (ACRS) to determine the areas of common interest of the two Committees; and (3) to discuss potential overlap of on-going activities of the ACRS and NSRRC Committee and coordinate these activities to ensure that areas of joint interest are supportive and complimentary and not duplicative. As time permits, a discussion will be initiated on the core technical competence to be maintained by the NRC's Office of Research staff.

Participants in parts of the discussion will include senior NRC staff and other RES technical staff as necessary.

Members of the public may file written statements regarding any matter to be discussed at the meeting. Members of the public may also make requests to speak at the meeting, but permission to speak will be determined by the Committee chairperson in accordance with procedures established by the Committee. A verbatim transcription will be made of the NSRRC meeting and a copy of the transcript will be placed in the NRC's Public Document Room in Washington, DC.

Any inquiries regarding this notice or any subsequent changes in the status and schedule of the meeting, may be made to the Designated Federal Officer, Dr. Jose Luis M. Cortez (telephone: 301–415–6596), between 8:15 am and 5:00 pm.

Dated at Rockville, Maryland this 7th day of November 1996.

For the Nuclear Regulatory Commission.

Andrew L. Bates,

Federal Advisory Committee Management Officer.

[FR Doc. 96–29153 Filed 11–13–96; 8:45 am]

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Individual Plant Examination Program: Perspectives on Reactor Safety and Plant Performance, Summary Report, Draft

AGENCY: Nuclear Regulatory Commission.

ACTION: Availability of NUREG, draft for public comment.

SUMMARY: The Nuclear Regulatory Commission has published a draft of ''Individual Plant Examination Program: Perspectives on Reactor Safety and Plant Performance, Summary Report,'' NUREG–1560, Volume 1, Part 1. This volume summarizes the insights and findings from a review of the Individual Plant Examinations (IPE) submitted to the agency in response to Generic Letter 88–20.

SUPPLEMENTARY INFORMATION: Draft NUREG–1560 (Volume 1, Part 1) is available for inspection and copying for a fee at the NRC Public Document Room, 2120 L Street N.W. (Lower Level), Washington D.C. 20555–0001. A free single copy of Draft NUREG–1560 (Volume 1, Part 1), to the extent of supply, may be requested by writing to Distribution Series, Printing and Mail Services Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

Draft NUREG–1560 provides perspectives gained from the review of the IPEs submitted in response to Generic Letter 88–20. Five major objectives were pursued in documenting perspectives from the reviews:

(1) The impact of the IPE program on reactor safety—

- The number and type of vulnerabilities or other safety issues that have been identified, and the related safety enhancements that have been implemented,
- The impact that the improvements have had on plant safety, and
- Whether any of these improvements have ''generic'' implications for all or a class of plants.

(2) Plant-specific features and assumptions that play a significant role in the estimation of core damage frequency (CDF) and the analysis of containment performance—

- Important design and operational features that affect CDF and containment performance, with regard to the different reactor and containment types,
- The influence of the IPE methodology and assumptions on the results, with regard to the different reactor and containment types, and
- Significant plant improvements to reduce CDF and increase containment performance, with regard to the different reactor and containment types.

(3) The importance of the operator's role in CDF estimation and containment performance analysis—

- Operator actions that are consistently important in the IPEs,
- Operator actions that are important because of plant-specific characteristics, and
- Influence of modeling assumptions and different methodologies on the results.

(4) IPEs with respect to risk-informed regulation—

- Quality of the IPEs, given the limited scope of the staff's review, compared to a quality probabilistic risk assessment, and therefore, the potential role of the IPEs in risk-informed regulation.

(5) General Perspectives—

- The implication of the IPE results relative to the current risk level of U.S. plants compared with the Commission's Safety Goals,
- The improvements that have been identified as a result of the Station Blackout Rule and analyzed as part of the IPE, and the impact of these improvements on reducing the likelihood of station blackout,
- The results of the IPEs compared with the perspectives gained from NUREG–1150.

Draft NUREG–1560 also documents the staff's preliminary overall conclusions and observations gained from the perspectives of each of the above noted areas. These conclusions and observations address the following:

- Generic Letter 88–20 objective (including improvement of plant safety).
- Regulatory follow-up activities:

—Plant safety enhancements,
—Containment performance improvements,
—Additional review of IPE/PRA,
—Plants with relatively high CDF or conditional containment failure probability.