N–588 are conservative and provide a margin of safety in the development of RPV P–T operating and pressure test limits that will prevent nonductile fracture of the vessel.

The staff concurs with the licensee's conclusion that the postulation of an axially oriented flaw on a circumferential RPV weld is a level of conservatism that is not required to establish P–T limits to protect the reactor coolant system pressure boundary from failure during hydrostatic testing, heatup, and cooldown. Based on the manufacturing processes used to fabricate RPVs for U.S. facilities, it is reasonable to conclude that, if a significant defect were to exist in a circumferential weld, it would lie in the plane of the welding direction. The use of stress magnification factors which account for this difference in flaw orientation (i.e., account for a factor of approximately two in the difference in the applied pressure stress between the axial and circumferential directions) is acceptable.

The staff also notes that, Code Case N–588, Section 2214.3, includes changes to the methodology for determining the thermal stress intensity, KIT, which was incorporated into Section XI of the ASME Code after the 1989 Edition. The staff has reviewed the basis for these changes in the KIT methodology in detail. The staff accepts that the modifications made to the KIT methodology in Section 2214.3 of Code Case N–588 result in a determination of KIT that is consistent with the methodology found in the 1989 Edition of ASME Code Section XI, Appendix G, and that the use of equivalent KIT values for axial and circumferential flaws is acceptable.

Application of ASME Code Case N–588 when determining P–T operating limit curves per ASME Code, Section XI, appendix G, provides appropriate procedures for determining limiting maximum postulated defects and considering those defects in developing the P–T limits. This application of the code case maintains that margin of safety originally contemplated when ASME Code Section XI, appendix G was developed.

Based on the above considerations, the staff concludes that use of Code Case N–588 for development of the HCGS RPV P–T limit curves will meet the underlying purpose of Appendix G of 10 CFR part 50 with respect to protecting the integrity of the reactor coolant pressure boundary. In this case, since strict compliance with the requirements of 10 CFR 50.60(a) and 10 CFR part 50, appendix G, is not necessary to serve the overall intent of the regulations, the staff also concludes that application of Code Case N–588 for the HCGS meets the special circumstances provisions in 10 CFR 50.12(a)(2)(ii), for granting exemptions to the regulations.

*Code Case N–640*

Code Case N–640 amends the provisions of ASME Section XI, Appendix G, by permitting the use of the $K_{Ic}$ equation as found in Appendix A in ASME Section XI, in lieu of the $K_{Ia}$ equation as found in Appendix G in ASME Section XI. Use of the $K_{Ic}$ equation in determining the lower bound fracture toughness in the development of the P–T operating limits curve is more technically correct than the use of the $K_{Ia}$ equation since the rate of loading during a heatup or cooldown is slow and is more representative of a static condition than a dynamic condition. The staff has required use of the initial conservatism of the $K_{Ia}$ equation since 1974 when the equation was codified. This initial conservatism was necessary due to the limited knowledge of RPV materials. Since 1974, additional knowledge has been gained about RPV materials, which demonstrates that the lower bound on fracture toughness provided by the $K_{Ia}$ equation is well beyond the margin of safety required to protect the public health and safety from potential RPV failure. In addition, P–T curves based on the $K_{Ic}$ equation will enhance overall plant safety by opening the P–T operating window with the greatest safety benefit in the region of low temperature operations.

Based on the above considerations, the staff concludes that use of Code Case N–640 for development of the HCGS RPV P–T limit curves will meet the underlying purpose of appendix G of 10 CFR part 50 with respect to protecting the integrity of the reactor coolant pressure boundary. In this case, since strict compliance with the requirements of 10 CFR 50.60(a) and 10 CFR part 50, appendix G, is not necessary to serve the overall intent of the regulations, the staff also concludes that application of Code Case N–640 for the HCGS meets the special circumstances provisions in 10 CFR 50.12(a)(2)(ii), for granting exemptions to the regulations.

## 4.0 Conclusion

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemption is authorized by law, will not endanger life or property or common defense and security, and is, otherwise, in the public interest. Also, special circumstances are present. Therefore, the Commission hereby grants PSEG Nuclear LLC an exemption from the requirements of 10 CFR 50.60(a) and 10 CFR part 50, appendix G, for HCGS.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will not have a significant effect on the quality of the human environment (66 FR 33717).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 12th day of July 2001.

For the Nuclear Regulatory Commission.

**John A. Zwolinski,**

*Director, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.*

[FR Doc. 01–17954 Filed 7–17–01; 8:45 am]

BILLING CODE 7590–01–P

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## NUCLEAR REGULATORY COMMISSION

### Public Meeting on Standard Review Plan for the Review of a License Application for a Fuel Cycle Facility

**AGENCY:** Nuclear Regulatory Commission (NRC).

**ACTION:** Notice of Meeting.

**SUMMARY:** NRC will host a public meeting in Rockville, Maryland. The meeting will provide an opportunity for discussion on the revised draft Chapter 3 entitled, ''Integrated Safety Analysis'' of NUREG–1520, Standard Review Plan (SRP) for the Review of a License Application for a Fuel Cycle Facility. The March 30, 2001, draft Chapter 3 can be found in both a ''clean'' and marked-up version in the NRC Public Electronic Reading Room under ''Recently Released Documents, April 3, 2001''. It can also be found on the Internet at the following website: *http://techconf.llnl.gov/cgi-bin/library?source=*&library=Part_70_lib.*

The web site can also be reached by the following method:

1. Go the main NRC web site at: *http://www.nrc.gov.*
2. Scroll down to the bottom of that page and click on the word ''Rulemaking.''
3. Scroll down on the Rulemaking page until the words ''Technical Conference'' appear. Click on those words.
4. On the page titled ''Welcome to the NRC Technical Conference Forum,'' click on the link ''Conference'' or ''Technical Conferences''.
5. Scroll down to the topic ''Draft Standard Review Plan and Guidance on Amendment to 10 CFR part 70.''
6. Select ''Document Library''.

*Purpose:* This meeting will provide an opportunity to discuss comments on the staff's revised draft Chapter 3 and its appendix.

**DATES:** The meeting is scheduled for Tuesday, August 2, 2001, from 1:30 p.m. to 4:00 p.m. The meeting is open to the public.

**ADDRESSES:** Two White Flint North, 11545 Rockville Pike, Room T–10A1, Rockville, Maryland. Visitor parking around the NRC building is limited; however, the meeting site is located adjacent to the White Flint Station on the Metro Red Line.

**FOR FURTHER INFORMATION CONTACT:** Yawar H. Faraz, Senior Project Manager, Fuel Cycle Licensing Branch, Division of Fuel Cycle Safety and Safeguards, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone: (301) 415–8113, e-mail *yhf@nrc.gov.*

Dated at Rockville, Maryland this 11th day of July, 2001.

For the Nuclear Regulatory Commission.

**Daniel M. Gillen,**

*Acting Chief, Fuel Cycle Licensing Branch, Division of Fuel Cycle Safety and Safeguards, Office of Nuclear Material Safety and Safeguards.*

[FR Doc. 01–17953 Filed 7–17–01; 8:45 am]

BILLING CODE 7590–01–P

## NUCLEAR REGULATORY COMMISSION

### Draft Regulatory Guide; Issuance, Availability

The Nuclear Regulatory Commission has issued for public comment a proposed revision of a regulatory guide in its Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods acceptable to the NRC staff for implementing specific parts of the NRC's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

Draft Regulatory Guide DG–1110 is a proposed Revision 1 of Regulatory Guide 1.174, ''An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis.'' DG–1110 is being developed to provide guidance to licensees on methods acceptable to the NRC staff for assessing the nature and impact of licensing basis changes when the licensee chooses to support, or is requested by the NRC staff to support, such changes with risk information.

A proposed Revision 1 of Chapter 19, ''Use of Probabilistic Risk Assessment in Plant-Specific, Risk-Informed Decisionmaking: General Guidance,'' of NUREG–0800, ''Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants,'' is being issued for public comment as a companion document. Chapter 19 of the Standard Review Plan will be used by the NRC staff for evaluating licensee submittals that use the guidance in Regulatory Guide 1.174 on risk-informed decisionmaking that uses probabilistic risk assessment.

This draft guide and draft standard review plan chapter have not received complete staff approval and do not represent an official NRC staff position.

Comments may be accompanied by relevant information or supporting data. Written comments may be submitted to the Rules and Directives Branch, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Copies of comments received may be examined at the NRC Public Document Room, 11555 Rockville Pike, Rockville, MD. Comments will be most helpful if received by September 17, 2001.

You may also provide comments via the NRC's interactive rulemaking web site through the NRC home page (http://www.nrc.gov). This site provides the ability to upload comments as files (any format) if your web browser supports that function. For information about the interactive rulemaking web site, contact Ms. Carol Gallagher, (301) 415–5905; e-mail *cag@nrc.gov.* For information about the draft guide and the related standard review plan chapter, contact Ms. M.T. Drouin at (301) 415–6675; e-mail mxd@nrc.gov.

Although a time limit is given for comments on this draft guide, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Electronic copies of this draft regulatory guide are available on the NRC's web site <*www.nrc.gov*> in the Reference Library under Regulatory Guides. Electronic copies are also available in NRC's Public Electronic Reading Room at the same web site; DG–1110 is under ADAMS Accession Number ML011770102. Regulatory guides are available for inspection at the NRC's Public Document Room, 11555 Rockville Pike, Rockville, MD; the PDR's mailing address is USNRC PDR, Washington, DC 20555; telephone (301) 415–4737 or (800) 397–4205; fax (301) 415–3548; email *pdr@nrc>gov.* Requests for single copies of draft or final guides (which may be reproduced) or for placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Reproduction and Distribution Services Section; or by e-mail to *distribution@nrc.gov*; or by fax to (301) 415–2289. Telephone requests cannot be accommodated. Regulatory guides are not copyrighted, and Commission approval is not required to reproduce them.

(5 U.S.C. 552(a))

Dated at Rockville, Maryland, this 27th day of June 2001.

For the Nuclear Regulatory Commission.

**Mark A. Cunningham,**

*Branch Chief, Probabilistic Risk Analysis Branch, Division of Risk Analysis and Applications, Office of Nuclear Regulatory Research.*

[FR Doc. 01–17955 Filed 7–17–01; 8:45 am]

BILLING CODE 7590–01–P

## POSTAL RATE COMMISSION

### Sunshine Act Meeting

**NAME OF AGENCY:** Postal Rate Commission.

**TIME AND DATE:** July 26, 2001 at 8:30 a.m.

**PLACE:** Commission conference room, 1333 H Street, NW., Suite 300, Washington, DC, 20268–0001.

**STATUS:** Open.

**MATTERS TO BE CONSIDERED:** Discussion and vote on the Postal Rate Commission's fiscal year 2002 budget.

**CONTACT PERSON FOR MORE INFORMATION:** Steven W. Williams, acting secretary, Postal Rate Commission, 202–789–6840.

Dated: July 16, 2001.

**Steven W. Williams,**

*Acting Secretary.*

[FR Doc. 01–18110 Filed 7–16–01; 2:09 pm]

BILLING CODE 7710–FW–M

## SECURITIES AND EXCHANGE COMMISSION

**[Investment Company Act Release No. 25061; 812–11616]**

### CDC IXIS Asset Management Advisers, L.P., et al.; Notice of Application

July 12, 2001.

**AGENCY:** Securities and Exchange Commission (''Commission'').

**ACTION:** Notice of an application for an order under sections 6(c) and 17(b) of