

Coating and Method for Application;
 NASA Case No. LAR-14240-1: Vacuum Holding Fixture for Fabricating Piezoelectric Acoustic Sensors;
 NASA Case No. LAR-14047-3: Method and Apparatus for Three-Dimensional Braiding;
 NASA Case No. LAR-13950-2: IBM Printer Port Interface;
 NASA Case No. LAR-13922-1: Apparatus for Use in Determining Surface Conductivity at Microwave Frequencies;
 NASA Case No. LAR-13890-1: Capacitive Acoustic Wave Detector and Method of Making Same;

Dated: July 22, 1996.

Edward A. Frankle,
 General Counsel.

[FR Doc. 96-19338 Filed 7-29-96; 8:45 am]

BILLING CODE 7510-01-M

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-387 and 50-388]

Pennsylvania Power and Light Company; Notice of Partial Withdrawal of Application for Amendment to Facility Operating Licenses

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Pennsylvania Power and Light Company (the licensee) to withdraw a portion of its application dated April 5, 1994, as supplemented on October 20, 1995, for proposed amendment to Facility Operating License Nos. NPF-14 and NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2, respectively, located in Luzerne County, Pennsylvania.

The portion of the proposed amendment which has been withdrawn would have revised the units' technical specifications by removing the specified frequency for Susquehanna Review Committee audits of the fire protection program at the site and would have had them listed in the Final Safety Analysis Report instead.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the Federal Register on May 25, 1994 (59 FR 27061). However, by letter dated July 5, 1996, the licensee withdrew the above portion of the proposed change.

For further details with respect to this action, see the application for amendment dated April 5, 1994, a supplemental letter dated October 20, 1995, and the licensee's letter dated July 5, 1996, which withdrew the above portion of the application for license

amendment. The above documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes-Barre, PA 18701.

Dated at Rockville, Maryland, this 24th day of July 1996.

For the Nuclear Regulatory Commission,
 Chester Posluny,
 Senior Project Manager, Project Directorate I-2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 96-19320 Filed 7-29-96; 8:45 am]

BILLING CODE 7590-01-P

[Docket Nos. 50-315 and 50-316]

Indiana Michigan Power Company; Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2 Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of 10 CFR 70.24 for Facility Operating License Nos. DPR-58 and DPR-74, issued to Indiana Michigan Power Company, (the licensee), for operation of the D. C. Cook Nuclear Plant, Units 1 and 2, located in Berrien County, Michigan.

Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt the licensee from the requirements of 10 CFR 70.24, which requires a monitoring system that will energize clearly audible alarms if accidental criticality occurs in each area in which special nuclear material is handled, used, or stored. The proposed action would also exempt the licensee from the requirements of 10 CFR 70.24(a)(3) to maintain emergency procedures for each area in which this licensed special nuclear material is handled, used, or stored to ensure that all personnel withdraw to an area of safety upon the sounding of the alarm and to conduct drills and designate responsible individuals for such emergency procedures.

This environmental assessment has been prepared to address potential environmental issues related to the licensee's application of April 8, 1996.

The Need for the Proposed Action

Power reactor license applicants are evaluated for the safe handling, use, and storage of special nuclear materials. The proposed exemption from criticality

accident requirements is based on the original design for fuel storage and handling at the D. C. Cook Nuclear Plant, Units 1 and 2. The exemption was granted with the original Unit 2 Special Nuclear Material (Part 70) license, but it expired with the issuance of the Part 50 license when the exemption was inadvertently not included in that license. Therefore, the exemption is needed to clearly define the design of the plant as evaluated and approved for licensing.

Environmental Impacts of the Proposed Action

The NRC staff has completed its evaluation of the proposed action and concludes that there is no significant environmental impact if the exemption is granted. Inadvertent or accidental criticality will be precluded through compliance with the Cook Technical Specifications, the geometric spacing of fuel assemblies in the new fuel storage facility and spent fuel storage pool, and administrative controls imposed on fuel handling procedures. Technical specification controls include reactivity requirements (e.g., shutdown margins, limits on control rod movement), instrumentation requirements (e.g., power and radiation monitors), and controls on refueling operations (e.g., refueling boron concentration and source range monitor requirements.) Geometrically, the spent fuel pool is designed to store the fuel in an array that precludes criticality. Existing technical specifications require the effective neutron multiplication factor, K_{eff} , to be maintained less than or equal to 0.95. The new fuel vault has also been analyzed to maintain k_{eff} less than or equal to 0.95, including uncertainties, under full water density flooded conditions and less than or equal to 0.98 under optimum moderation conditions.

In summary, the training provided to all personnel involved in fuel handling operations, the design of the fuel handling equipment, the administrative controls, the technical specifications on new and spent fuel handling and storage, and the design of the new and spent fuel storage racks preclude inadvertent or accidental criticality. In accordance with the NRC's Regulatory Position in Regulatory Guide 8.12, Revision 1, "Criticality Accident Alarm Systems," dated January 1981, an exemption from 10 CFR 70.24 is appropriate.

The proposed exemption will not affect radiological plant effluents nor cause any significant occupational exposures. Only a small amount, if any, radioactive waste is generated during the receipt and handling of new fuel