sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, and to Douglas K. Porter, Esquire, Southern California Edison Company, 2244 Walnut Grove Avenue, Rosemead, California 91770, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)–(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated April 11, 1996, as supplemented April 6, 1998, March 22, and July 29, 1999, which are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and accessible electronically through the ADAMS Public Electronic Reading Room link at the NRC Web site (http://www.nrc.gov).

Dated at Rockville, Maryland, this 12th day of January 2000.

For the Nuclear Regulatory Commission.

L. Raghavan,

Senior Project Manager, Section 2, Project Directorate IV & Decommissioning, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 00–1175 Filed 1–18–00; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket 72–1014]

Holtec International; Issuance of Environmental Assessment and Finding of No Significant Impact Regarding the Request for Exemption From Requirements of 10 CFR Part 72

By letter dated November 15, 1999, Holtec International (Holtec or applicant) requested an exemption, pursuant to 10 CFR 72.7, from the requirements of 10 CFR 72.234(c). Holtec, located in Marlton, New Jersey, is seeking Nuclear Regulatory Commission (NRC or the Commission) approval to fabricate four HI–STORM 100 overpacks, and one HI–TRAC 100 transfer cask prior to receipt of the Certificate of Compliance (CoC) for the HI–STORM 100 cask system. The HI–STORM 100 overpack and the HI–TRAC 100 transfer cask are basic components of the HI–STORM 100 system, a cask system designed for the dry storage and transportation of spent nuclear fuel. The HI–STORM 100 cask system is intended for use under the general license provisions of Subpart K of 10 CFR Part 72 by Commonwealth Edison Company (ComEd) at the Dresden Nuclear Power Station, Units 2 and 3 (Dresden), located in Morris, Illinois.

Environmental Assessment (EA)

Identification of Proposed Action

By letter dated October 26, 1995, as supplemented, and pursuant to 10 CFR Part 72, Holtec submitted an application to the NRC for a CoC for the HI–STORM 100 cask system. This application is currently under consideration by the NRC staff. The applicant is seeking Commission approval to fabricate four HI–STORM 100 overpacks and one HI–STORM 100 transfer cask prior to the Commission's issuance of a CoC for the HI–STORM 100 cask system. The HI–STORM 100 cask system is intended for use under the general license provisions of Subpart K of 10 CFR Part 72 by ComEd at Dresden in Morris, Illinois. The applicant requests an exemption from the requirements of 10 CFR 72.234(c), which state that ''Fabrication of casks under the Certificate of Compliance must not start prior to receipt of the Certificate of Compliance for the cask model.'' The proposed action before the Commission is whether to approve fabrication, including material procurement, and whether to grant this exemption pursuant to 10 CFR 72.7.

Need for the Proposed Action

Holtec requested the exemption to 10 CFR 72.234(c) to ensure the availability of overpacks so that ComEd can maintain full core off-load capability at Dresden. Dresden will lose full core off-load capability in the fall of 2001. Dresden requests the delivery of the four HI–STORM 100 overpacks and one HI–TRAC 100 transfer cask by November 20, 2001. Holtec states that to meet this schedule, fabrication must begin by February 15, 2000.

The HI–STORM 100 cask system application, dated October 26, 1995, is under consideration by the Commission. It is anticipated that, if approved, the HISTORM–100 cask system CoC may be issued by July 2000. The proposed fabrication exemption will not authorize use of any Holtec overpack to store spent fuel. That will occur only when, and if, a CoC is issued. An NRC approval of the fabrication exemption request should not be construed as an NRC commitment to favorably consider any Holtec application for a CoC. Holtec will bear the risk of all activities conducted under the exemption, including the risk that the four HI–STORM 100 overpacks and one HI–TRAC 100 transfer cask that Holtec plans to construct may not be usable because they may not meet specifications or conditions placed in a CoC that the NRC may ultimately approve.

Environmental Impacts of the Proposed Action

Regarding the fabrication exemption, the Environmental Assessment for the final rule, ''Storage of Spent Nuclear Fuel in NRC-Approved Storage Casks at Nuclear Power Reactor Sites' (55 FR 29181 (1990)), considered the potential environmental impacts of overpacks which are used to store spent nuclear fuel under a CoC and concluded that there would be no significant environmental impacts. The proposed action now under consideration would not permit use of the overpacks, but would only permit fabrication. There are no radiological environmental impacts from fabrication since overpack fabrication does not involve radioactive materials. The major non-radiological environmental impacts involve use of natural resources due to overpack fabrication. Each HI–STORM 100 overpack weighs approximately 100 tons and is constructed of metal and concrete. The HI–TRAC 100 transfer cask weighs approximately 125 tons and is made of structural steel and lead. The amount of materials required to fabricate these components is expected to have very little impact on the associated industry. Fabrication of the metal components would be at a metal fabrication facility, while fabrication of the concrete overpacks would be partially fabricated at the same metal fabrication facility, with only the concrete pours being done at Dresden. The metal and concrete used in the fabrication of these components is insignificant compared to the amount of metal and concrete fabrication performed annually in the United States. If the components are not usable, the components could be disposed of or recycled. The amount of metal and concrete disposed of is insignificant compared to the amount of metal and concrete that is disposed of annually in the United States. Based upon this information, the fabrication of these components will have no significant impact on the environment since no radioactive materials are involved, and the amount of natural resources used is minimal.

Alternative to the Proposed Action

Since there is no significant environmental impact associated with the proposed actions, any alternatives with equal or greater environmental impact are not evaluated. The alternative to the proposed actions would be to deny approval of the exemption and, therefore, not allow fabrication until a CoC is issued. This alternative would have the same environmental impact.

Given that there are no significant differences in environmental impact between the proposed action and the alternative considered and that the applicant has a legitimate need to fabricate the components prior to certification and is willing to assume the risk that any fabricated components may not be approved or may require modification, the Commission concludes that the preferred alternative is to grant the exemption from the prohibition on fabrication prior to receipt of a CoC.

Agencies and Persons Consulted

Mr. F. Niziolek, Reactor Safety Section Head, Illinois Department of Nuclear Safety, was contacted about the Environmental Assessment for the proposed action and had no comments.

Finding of No Significant Impact

The environmental impacts of the proposed action have been reviewed in accordance with the requirements set forth in 10 CFR Part 51. Based upon the foregoing Environmental Assessment, the Commission finds that the proposed action of granting an exemption from 10 CFR 72.234(c) so that Holtec may fabricate four HI–STORM 100 overpacks and one HI–TRAC–100 transfer cask prior to issuance of a CoC will not significantly impact the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed exemption.

The request for the exemption from 10 CFR 72.234(c) was filed on November 15, 1999. For further details with respect to this action, see the application for CoC for the HI–STORM 100 cask system, dated October 26, 1995. On July 30, 1999, a preliminary Safety Evaluation Report and a proposed CoC for the HI–STORM 100 cask system were issued by the NRC staff to initiate the rulemaking process. The exemption request and CoC application are docketed under 10 CFR Part 72, Docket 72–1014. These documents are available for public inspection at the Commission's Public Document Room, 2120 L Street, NW, Washington, DC 20555.

Dated at Rockville, Maryland, this 10th day of January 2000.P='02'≤

For the Nuclear Regulatory Commission.

E. William Brach,

Director Spent Fuel Project, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 00–1173 Filed 1–18–00; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Experts' Meeting on High-Burnup Fuel Behavior Under Postulated Accident Conditions

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of Meeting.

SUMMARY: The Nuclear Regulatory Commission will hold a meeting to develop Phenomena Identification and Ranking Tables (PIRTs). PIRTs have been used at NRC since 1988, and they provide a structured way to obtain a technical understanding that is needed to address certain issues. About twenty of the world's best technical experts are participating in this activity, and the experts represent a balance between industry, universities, foreign researchers, and regulatory organizations. The current PIRT activity is addressing a postulated BWR accident wherein power oscillations occur, the reactor fails to scram, and the oscillations then reach sufficient magnitude that fuel failure may occur before the emergency operating procedures are able to terminate the oscillations and shut the reactor down.

DATES: February 8–10, 2000, 8:30 am–5:30 pm.

ADDRESSES: Room T10A1 (TWFN) of the Nuclear Regulatory Commission, 11545 Rockville Pike, Rockville, MD.

FOR FURTHER INFORMATION CONTACT: Dr. Ralph Meyer, SMSAB, Division of Systems Analysis and Regulatory Effectiveness, Office of Nuclear Regulatory Research, Washington, DC 20555–0001, telephone (301) 415–6789.

SUPPLEMENTARY INFORMATION: The meeting agenda will be posted on the NRC Web site at www.nrc.gov/RES/meetings.html by February 1, 2000. The meeting is open to the public. Attendees will need to obtain a visitor badge at the TWFN building lobby.

Dated at Rockville, Maryland, this 10th day of January 2000.

For the Nuclear Regulatory Commission.

Charles E. Rossi,

Director, Division of Systems Analysis and Regulatory Effectiveness, Office of Nuclear Regulatory Research.

[FR Doc. 00–1176 Filed 1–18–00; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

Sunshine Act Meeting

AGENCY HOLDING THE MEETING: Nuclear Regulatory Commission.P='02'≤