

June 30, 2001, this Order shall become null and void, provided, however, on application and for good cause shown, such date may be extended.

This Order is effective upon issuance.

For further details with respect to this action, see the initial application dated February 17, 2000, and supplements thereto dated March 1, April 24, April 28, and May 10, 2000, and the Safety Evaluation dated July 18, 2000, which are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC and accessible electronically through the ADAMS Public Electronic Reading Room link at the NRC Web site (<http://www.NRC.gov>).

For the Nuclear Regulatory Commission.

Dated at Rockville, Maryland, this 18th day of July, 2000.

Samuel J. Collins,

Director, Office of Nuclear Reactor Regulation.

[FR Doc. 00-18655 Filed 7-21-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

U.S. Nuclear Regulatory Commission Seeks Qualified Candidates for the Advisory Committee on Reactor Safeguards

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Request for resumes.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is seeking two qualified candidates for appointment to its Advisory Committee on Reactor Safeguards (ACRS).

ADDRESSES: Submit resumes to: Ms. Robin Avent, Office of Human Resources, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

For Application Materials, Call: 1-800-952-9678. Please refer to Announcement Number 60000001.

SUPPLEMENTARY INFORMATION: Congress established the ACRS to provide the NRC with independent expert advice on matters related to licensing and the safety of existing and proposed nuclear power plants. The Committee's work currently emphasizes safety issues associated with the operation of 103 commercial nuclear power plants in the United States; the pursuit of a risk-informed, and performance-based regulatory approach; review of license renewal applications; digital instrumentation and control systems;

and technical issues related to standard plant designs.

The ACRS membership includes individuals from national laboratories, academia, and industry who possess specific technical expertise along with a broad perspective in addressing safety concerns. Committee members are selected from a variety of engineering and scientific disciplines, such as nuclear power plant operations, nuclear engineering, mechanical engineering, electrical engineering, chemical engineering, metallurgical engineering, structural engineering, materials science, and instrumentation and process control systems. At this time, candidates are specifically being sought who have 15-20 years of experience, including graduate level education, in the areas of structural mechanics/materials engineering and metallurgy applicable to nuclear power systems, and the application of risk methods related to nuclear regulatory safety issues.

Criteria used to evaluate candidates include education and experience, demonstrated skills in nuclear reactor matters, and the ability to solve problems. Additionally, the Commission considers the need for specific expertise in relationship to current and future tasks. Consistent with the requirements of the Federal Advisory Committee Act, the Commission seeks candidates with diverse viewpoints so that the membership on the Committee will be fairly balanced.

Because conflict-of-interest regulations restrict the participation of members actively involved in the regulated aspects of the nuclear industry, the degree and nature of any such involvement will be weighed. Each qualified candidate's financial interests must be reconciled with applicable Federal and NRC rules and regulations prior to final appointment. This might require divestiture of securities issued by nuclear industry entities, or discontinuance of industry-funded research contracts or grants.

Copies of a resume describing the educational and professional background of the candidate, including any special accomplishments, professional references, current address and telephone number should be provided. All qualified candidates will receive careful consideration. Appointment will be made without regard to such factors as race, color, religion, national origin, sex, age, or disabilities. Candidates must be citizens of the United States and be able to devote approximately 60-100 days per year to Committee business.

Applications will be accepted until September 29, 2000.

Dated: July 18, 2000.

Andrew L. Bates,

Advisory Committee Management Officer.

[FR Doc. 00-18653 Filed 7-21-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[DOCKET NO. 50-400]

Carolina Power & Light Company; Shearon Harris Nuclear Power Plant, Unit 1, Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from certain requirements of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.60(a) for Facility Operating License No. NPF-63, issued to Carolina Power & Light Company (CP&L, the licensee) for operation of the Shearon Harris Nuclear Power Plant, Unit 1 (HNP), located in Wake and Chatham Counties, North Carolina.

Environmental Assessment

Identification of the Proposed Action

10 CFR Part 50, Appendix G, requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak testing conditions. Specifically, 10 CFR Part 50, Appendix G, states that, "[t]he appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions." Appendix G of 10 CFR Part 50 specifies that the requirements for these limits are the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G Limits.

To address provisions of amendments to the technical specifications (TS) P-T limits and low temperature overpressure protection (LTOP) system setpoints, the licensee requested in its submittal dated April 12, 2000, as supplemented on June 2, 2000, that the staff exempt HNP from application of specific requirements of 10 CFR Part 50, Section 50.60(a) and Appendix G, and substitute use of ASME Code Case N-640. Code Case N-640 permits the use of an alternate reference fracture toughness (K_{IC} fracture toughness curve instead of K_{Ia} fracture toughness curve) for reactor vessel materials in determining the P-T limits and LTOP setpoints. Since the K_{IC} fracture toughness curve shown in

ASME Section XI, Appendix A, Figure A-2200-1 (the K_{Ic} fracture toughness curve) provides greater allowable fracture toughness than the corresponding K_{Ia} fracture toughness curve of ASME Section XI, Appendix G, Figure G-2210-1 (the K_{Ia} fracture toughness curve), using Code Case N-640 for establishing the P-T limits and LTOP setpoints would be less conservative than the methodology currently endorsed by 10 CFR Part 50, Appendix G and, therefore, an exemption to apply the Code Case would be required by 10 CFR 50.60. It should be noted that, although Code Case N-640 was incorporated into the ASME Code recently, an exemption is still needed because the proposed P-T limits and LTOP setpoints (excluding Code Cases N-640) are based on the 1989 edition of the ASME Code.

The proposed action is in accordance with the licensee's application for exemption dated April 12, 2000, as supplemented on June 2, 2000.

The Need for the Proposed Action

Use of the K_{Ic} curve, Code Case N-640, in determining the lower bound fracture toughness in the development of P-T operating limit curves and LTOP setpoints is more technically correct than use of the K_{Ia} curve since the rate of loading during a heatup or cooldown is slow and is more representative of a static condition than a dynamic condition. The K_{Ic} curve appropriately implements the use of static initiation fracture toughness behavior to evaluate the controlled heatup and cooldown process of a reactor vessel. The staff has required use of the conservatism of the K_{Ia} curve since 1974, when the curve was adopted by the ASME Code. This conservatism was initially necessary due to the limited knowledge of the fracture toughness of RPV materials at that time. Since 1974, additional knowledge has been gained about RPV materials, which demonstrates that the lower bound on fracture toughness provided by the K_{Ia} curve greatly exceeds the margin of safety required to protect the public health and safety from potential RPV failure. In addition, P-T curves and LTOP setpoints based on the K_{Ic} curve will enhance overall plant safety by opening the P-T operating window, with the greatest safety benefit in the region of low temperature operations.

Since an unnecessarily reduced P-T operating window can reduce operator flexibility without just basis, implementation of the proposed P-T curves and LTOP setpoints as allowed by ASME Code Case N-640 may result in enhanced safety during critical plant operational periods, specifically heatup

and cooldown conditions. Thus, pursuant to 10 CFR 50.12(a)(2)(ii), the underlying purpose of 10 CFR 50.60 and Appendix G to 10 CFR Part 50 will continue to be served.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that the exemption described above would provide an adequate margin of safety against brittle failure of the HNP reactor pressure vessel.

The proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological environmental impacts, the proposed action does not involve any historic sites. It does not affect nonradiological plant effluents and has no other environmental impacts.

Therefore, there are no significant nonradiological impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (*i.e.*, the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for HNP.

Agencies and Persons Consulted

In accordance with its stated policy, on July 11, 2000, the staff consulted with the North Carolina State official, Mr. Johnny James of the North Carolina Department of Environment and Natural Resources, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the

human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated April 12, 2000, as supplemented on June 2, 2000, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC. Publicly available records will be accessible electronically from the ADAMS Public Library component on the NRC Web site, <http://www.nrc.gov> (the Electronic Reading Room).

Dated at Rockville, Maryland, this 17th day of July 2000.

For the Nuclear Regulatory Commission.

Richard J. Laufer,

Project Manager, Section 2, Project Directorate II, Division of Licensing Project Management Office of Nuclear Reactor Regulation.

[FR Doc. 00-18656 Filed 7-21-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

NRC To Hold Public Meetings on Spent Fuel Shipping Cask Accident Studies

AGENCY: U.S. Nuclear Regulatory Commission.

ACTION: Notice of public meetings on spent nuclear fuel transportation studies.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC or Commission) is approaching the end of the scoping phase of a study on spent nuclear fuel cask responses to severe transportation accidents (*i.e.*, the Package Performance Study (PPS)). The scoping phase will determine which issues and approaches are to be used for succeeding phases (including planning, conducting, and documenting any analyses or tests). In addition, in March 2000, NRC published the technical report for a related study, NUREG/CR-6672, "Reexamination of Spent Fuel Shipment Risk Estimates," and a discussion summary paper is available to complement that technical report. To facilitate discussion on these activities, NRC is convening an August public workshop and two public meetings in Nevada, and a September workshop in Rockville, Maryland.

A World Wide Web site has been established for dissemination of information to interested members of the public. Electronic copies of documents related to these studies, and