Estimated Annual Burden Per Voluntary Respondent: 30 hours.

Estimated Total Annual Burden on Respondents: 150 hours.

Jacqueline Agtuca,

Chief of Staff.

[FR Doc. 01–10917 Filed 5–1–01; 8:45 am]

BILLING CODE 7565–01–P

NORTHEAST DAIRY COMPACT COMMISSION

Notice of Meeting

AGENCY: Northeast Dairy Compact Commission.

ACTION: Notice of meeting.

SUMMARY: The Compact Commission will hold its regular monthly meeting to consider matters relating to administration and enforcement of the price regulation. This meeting will be held in Rhode Island, continuing the Commission's program of holding a meeting in each of the Compact states. In addition to receiving reports and recommendations of its standing Committees, the Commission will receive a number of informational reports, including reports on the operation of the wholesale and retail markets and about the impact of the price regulation on the Rhode Island WIC Program.

DATES: The meeting will begin at 10 a.m. on Friday, May 11, 2001.

ADDRESSES: The meeting will be held at the Newport Marriott Hotel, 25 America's Cup Avenue, Newport, Rhode Island.

FOR FURTHER INFORMATION CONTACT: Daniel Smith, Executive Director, Northeast Dairy Compact Commission, 64 Main Street, Room 21, Montpelier, VT 05602. Telephone (802) 229–1941.

Authority: 7 U.S.C. 7256.

Dated: April 25, 2001.

Daniel Smith,

Executive Director.

[FR Doc. 01–10888 Filed 5–1–01; 8:45 am]

BILLING CODE 1650–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50–338 and 50–339]

Virginia Electric and Power Company, North Anna Power Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from the requirements of Title 10 of the Code of Federal Regulations (10 CFR) part 50, appendix G, for Facility Operating License Nos. NPF–4 and NPF–7, issued to Virginia Electric and Power Company (the licensee), for operation of the North Anna Power Station, Units 1 and 2, located in Louisa County, Virginia.

Environmental Assessment

Identification of the Proposed Action

10 CFR Part 50, Appendix G, requires that the pressure-temperature (P–T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak testing conditions. Specifically, 10 CFR part 50, Appendix G, states that ''[t]he appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions.'' Appendix G of 10 CFR part 50 specifies that the requirements for these limits are contained in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G.

To address provisions of an amendment to the Technical Specifications P–T limits and low-temperature overpressure protection (LTOP) system setpoints, the licensee requested in its submittal dated June 22, 2000, as supplemented on January 4, February 14, March 13, and March 22, 2001, that the NRC staff exempt North Anna Power Station from the requirements of 10 CFR Part 50, Appendix G, to allow the use of ASME Code Case N–641.

Code Case N–641 permits the use of an alternate reference fracture toughness (K_{IC} fracture toughness curve instead of the K_{IA} fracture toughness curve) for reactor vessel materials in determining the P–T limits, LTOP system setpoints and T_{enable}, and provides for plant-specific evaluation of T_{enable}. Since the K_{IC} fracture toughness curve shown in ASME Section XI, Appendix A, Figure A–2200–1 (the K_{IC} fracture toughness curve) provides greater allowable fracture toughness than the corresponding K_{Ia} fracture toughness curve of ASME Section XI, Appendix G, Figure G–2210–1 (the K_{Ia} fracture toughness curve), and a plant-specific evaluation of T_{enable} would give lower values of T_{enable} than use of a generic bounding evaluation for T_{enable}, use of Code Case N–641 for establishing the P–T limits, LTOP system setpoints and T_{enable} would be less conservative than the methodology currently endorsed by 10 CFR Part 50, Appendix G. Although the use of the K_{IC} fracture toughness curve in ASME Code Case N–641 was recently incorporated into Appendix G to Section XI of the ASME Code, an exemption is still needed because 10 CFR Part 50, Appendix G requires a licensee's analysis to use an edition and addenda of Section XI of the ASME Code incorporated by reference into 10 CFR Part 50, section 50.55a, i.e., the editions through 1995 and addenda through the 1996 addenda (which do not include the provisions of Code Case N–641). Therefore, an exemption to apply the Code case is required by 10 CFR Part 50, section 50.60. The proposed action is in accordance with the licensee's application for exemption dated June 22, 2000, as supplemented by letters dated January 4, February 14, March 13, and March 22, 2001.

The Need for the Proposed Action

ASME Code Case N–641 is needed to revise the method used to determine the reactor coolant system (RCS) P–T limits, LTOP setpoints, and T_{enable}.

The purpose of 10 CFR part 50, Section 50.60(a), and 10 CFR part 50, appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. Pursuant to 10 CFR part 50, appendix G, it is required that P–T limits for the RCS be at least as conservative as those obtained by applying the methodology of the ASME Code, Section XI, Appendix G.

Current overpressure protection system (OPPS) setpoints produce operational constraints by limiting the P–T range available to the operator to heat up or cool down the plant. The operating window through which the operator heats up and cools down the RCS becomes more restrictive with continued reactor vessel service. Reducing this operating window could potentially have an adverse safety impact by increasing the possibility of inadvertent OPPS actuation due to pressure surges associated with normal plant evolutions such as reactor coolant pump start and swapping operating charging pumps with the RCS in a water-solid condition. The impact on the P–T limits and OPPS setpoints has been evaluated for an increased service period for operation to 32.3 effective full-power years (EFPYs) for Unit 1 and 34.3 EFPYs for Unit 2, based on ASME Code, Section XI, Appendix G requirements. The results indicate that these OPPS setpoints would significantly restrict the ability to perform plant heatup and cooldown,