For further details with respect to the proposed action, see the licensee’s letters dated January 29, 2010. Documents may be examined, and/or copied for a fee, at the NRC’s Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, http://www.nrc.gov/reading-rm/adams.html. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1–800–397–4209 or 301–415–4737, or send an e-mail to pdr.resource@nrc.gov.

Dated at Rockville, Maryland, this 1st day of December 2010.

For the Nuclear Regulatory Commission.

V. Sreenivas,
Project Manager, Plant Licensing Branch 2–1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 70–143; NRC–2010–0379]

Nuclear Fuel Services, Inc.; Environmental Assessment and Finding of No Significant Impact for Proposed Exemption From a Requirement To Measure the Uranium Element and Isotopic Content of Special Nuclear Material

AGENCY: Nuclear Regulatory Commission.

ACTION: Environmental Assessment and Finding of No Significant Impact.

FOR FURTHER INFORMATION CONTACT: Kevin M. Ramsey, Project Manager, Fuel Cycle Facilities Branch, Division of Fuel Cycle Safety and Safeguards, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Mail Stop EBB–2C40M, Rockville, MD 20855–0001, Telephone (301) 492–3123, Fax (301) 492–3359, E-mail kevin.ramsey@nrc.gov.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission’s (NRC) staff is considering the issuance of a license amendment to Materials License SNM–124 to Nuclear Fuel Services, Inc. (NFS or the licensee) that would reflect a requested one-time exemption from a requirement to measure the uranium element and isotopic content of certain small amounts of strategic special nuclear material, as described further below. The NRC regulations in Title 10 of the Code of Federal Regulations (10 CFR) 74.59(d)(1) state that a licensee must establish and maintain a system of measurements to substantiate such contents. By letter dated December 31, 2009, NFS requested a temporary exemption from this requirement.

The NRC prepared an Environmental Assessment (EA) in support of this exemption request in accordance with the requirements of 10 CFR part 51. Based on the EA, the NRC concluded that a Finding of No Significant Impact (FONSI) is appropriate; therefore, an Environmental Impact Statement (EIS) will not be prepared.

II. Environmental Assessment

Background

The NFS facility in Erwin, Tennessee is authorized, under License SNM–124 to manufacture high-enriched nuclear reactor fuel. In addition, NFS is authorized to blend highly enriched uranium with natural uranium and manufacture low-enriched nuclear reactor fuel. The U.S. Department of Energy contracted with NFS to retain no more than 30, 2S type uranium hexafluoride (UF₆) cylinders for future forensic analysis. These cylinders have been opened and processed leaving a small quantity of material (heel) in each cylinder. Because of the trace condition of the material, it is difficult to perform destructive or nondestructive analyses to measure the uranium element and isotopic content of the material remaining in these cylinders. It requires expensive equipment, which NFS does not possess, to sample and analyze UF₆ gas.

Alternatives

The alternatives available to NRC are:

1. Approve the requested action as described, or
2. No action (i.e., deny the request).

Affected Environment

The affected environment for the proposed action and the no action alternative is the NFS site. The NFS facility is located in Unicoi County, Tennessee, about 32 kilometers (20 miles) southwest of Johnson City, Tennessee. The facility is within the Erwin city limits. The affected environment is identical to the affected environment assessed in the 2002 EA related to the first amendment for the BLEU Project (Reference 2). A full description of the site and its characteristics are given in the 2002 EA. Additional information can be found in the 1999 EA related to the renewal of the NFS license (Reference 1). The site
occupies about 28 hectares (70 acres). The site is bounded to the northwest by the CSX Corporation (CSX) railroad property and the Nolichucky River; and by Martin Creek to the northeast. The plant elevation is about 9 meters (30 feet) above the nearest point on the Nolichucky River.

The area adjacent to the site consists primarily of residential, industrial, and commercial areas; with a limited amount of farming to the northwest. Privately owned residences are located to the east and south of the facility. Tract size is relatively large, leading to a low housing density in the areas adjacent to the facility. The CSX railroad right-of-way is parallel to the western boundary of the site. Industrial development is located adjacent to the railroad on the opposite side of the right-of-way. The site is bounded by Martin Creek to the north with privately owned, vacant property and low-density residences.

**Environmental Impacts of Proposed Action and Alternatives**

1. Occupational and Public Health Proposed Action

The occupational and public health impacts from the proposed action are essentially the same as those considered in the previous environmental assessments. If the exemption is granted, no samples of the radioactive and chemically hazardous material will be removed from the cylinders and measured in a laboratory, which will reduce the risk of exposures and releases from measurement operations and reduce the risk of accidents. However, the reductions would be so small that the differences would be negligible.

No Action

Denying the exemption request would not result in a significant difference in the occupational and public health impacts when compared to the proposed action. If this exemption request is denied, the licensee may make arrangements to have the material in each cylinder sampled and measured, which will increase the risk of exposures and releases from measurement operations and increase the risk of accidents. However, the facility will continue to implement NRC-approved procedures for handling radioactive and chemically hazardous materials. Thus, the impacts under the “no action” alternative will remain within acceptable regulatory limits. In addition, the quantity of material involved is relatively small. The increased risk would be so small that the difference would be negligible.

**Conclusion**

Based on its review, the NRC concluded that the environmental impacts associated with the proposed action are not significant and, therefore, do not warrant the preparation of an EIS. The NRC determined that the proposed action is the appropriate alternative for selection. Based on an evaluation of the environmental impacts of the proposed action, the NRC determined that the proper action is to issue a FONSI.

**Agencies and Persons Contacted**

On October 19, 2010, the NRC staff contacted the Division of Radiological Health in the Tennessee Department of Environment and Conservation (TDEC) concerning this EA. On November 15, 2010, TDEC responded that it had reviewed the draft EA and had no comments (Reference 6).

The NRC staff determined that the proposed action will not affect listed species or critical habitat. Therefore, no consultation is required under section 7 of the Endangered Species Act. Likewise, the NRC staff determined that the proposed action is not the type of activity that has the potential to cause effects on historic properties. Therefore, no consultation is required under section 106 of the National Historic Preservation Act.

**III. Finding of No Significant Impact**

Pursuant to 10 CFR part 51, the NRC staff considered the environmental consequences of taking the proposed action. On the basis of this EA, the NRC has concluded that there are no significant environmental impacts associated with the proposed action, and that preparation of an EIS is not warranted.

**IV. Further Information**

The documents referenced below in this Notice may be made available to interested parties, pursuant to a protective order and subject to applicable security requirements upon showing that the party has an interest that may be affected by the proposed action.

**References**

6. D. Shults, Director, Tennessee Division of Radiological Health, e-mail to K. Ramsey, U.S. Nuclear Regulatory Commission, “State Consultation on EA...

Dated at Rockville, Maryland, this 1st day of December 2010.

For the Nuclear Regulatory Commission.

Merritt Baker,

[FR Doc. 2010–30860 Filed 12–7–10; 8:45 am]
BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–346; NRC–2010–0378]

Firstenergy Nuclear Operating Company, Davis–Besse Nuclear Power Station; Environmental Assessment And Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC, or the Commission) is considering issuance of an Exemption, pursuant to Title 10 of the Code of Federal Regulations (10 CFR) Section 50.12, “Specific Exemptions,” from 10 CFR 50.61 “Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events” and from 10 CFR part 50, Appendix G, “Fracture Toughness Requirements” for Facility Operating License No. NPF–3, issued to FirstEnergy Nuclear Operating Company (FENOC, the licensee), for operation of the Davis–Besse Nuclear Power Station, Unit 1 (DBNPS), located in Ottawa County, Ohio. In accordance with 10 CFR 51.21, the NRC performed an environmental assessment documenting its findings. The NRC concluded that the proposed actions will have no significant environmental impact.

Environmental Assessment

Identification of the Proposed Action

Title 10 of the Code of Federal Regulations (10 CFR), Part 50, Appendix G requires that fracture toughness requirements for ferritic materials of pressure-retaining components of the reactor coolant pressure boundary of light-water nuclear power reactors provide adequate margins of safety during any condition of normal operation, including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected over its service lifetime, section 50.61 provides fracture toughness requirements for protection against pressurized thermal shock (PTS) events. By letter dated April 15, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML091130228), as supplemented by letter dated December 18, 2009 (ADAMS Accession No. ML093570103), and October 8, 2010 (ADAMS Accession No. ML102861221), FENOC proposed exemptions from the requirements of 10 CFR part 50, Appendix G and 10 CFR 50.61, to revise certain DBNPS reactor pressure vessel (RPV) initial (unirradiated) properties using Framatome Advanced Nuclear Power Topical Report BAW–2308, Revisions 1–A and 2–A, “Initial R\textsubscript{TPS} of Linde 80 Weld Materials.”

The licensee requested an exemption from Appendix G to 10 CFR part 50 to replace the required use of the existing Charpy V-notch (C\textsubscript{V}) and drop weight-based methodology and allow the use of an alternate methodology to incorporate the use of fracture toughness test data for evaluating the integrity of the DBNPS RPV circumferential beltl ine welds based on the use of the 1997 and 2002 editions of American Society for Testing and Materials (ASTM) Standard Test Method E 1921, “Standard Test Method for Determination of Reference Temperature T\textsubscript{ref} for Ferritic Steels in the Transition Range,” and American Society for Mechanical Engineering Boiler and Pressure Vessel Code (ASME B\&PV Code), Code Case N–629, “Use of Fracture Toughness Test Data to establish Reference Temperature for Pressure Retaining materials of Section III, Division 1, Class I.” The exemption is required since Appendix G to 10 CFR part 50, through reference to Appendix G to Section XI of the ASME Code, pursuant to 10 CFR 51.55(a), requires the use of a methodology based on \(C\textsubscript{V}\) and drop weight data.

The licensee also requested an exemption from 10 CFR 50.61 to use an alternate methodology to allow the use of fracture toughness test data for evaluating the integrity of the DBNPS RPV circumferential beltl ine welds based on the use of the 1997 and 2002 editions of ASTM E 1921 and ASME Code Case N–629. The exemption is required since the methodology for evaluating RPV material fracture toughness in 10 CFR 50.61 requires the use of the \(C\textsubscript{V}\) and drop weight data for establishing the PTS reference temperature (R\textsubscript{TPS}).

The proposed action is in accordance with the licensee’s application dated April 15, 2009, as supplemented by letters dated December 18, 2009, August 26 and October 8, 2010.

The Need for the Proposed Action

The proposed action is needed to allow the licensee to use an alternate method, as described in Topical Report BAW–2308, Revisions 1–A and 2–A, “Initial R\textsubscript{TPS} of Linde 80 Weld Materials” for determining the initial, unirradiated material reference temperatures of the Linde 80 weld materials present in the beltl ine region of the DBNPS RPV. This action, by being exempted from 10 CFR 50.61 would allow the licensee to revise its pressurized thermal shock reference temperature values in the future.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed exemption. The NRC staff has concluded that the proposed action to allow an alternate method for determining the initial, unirradiated material reference temperatures of the Linde 80 weld materials present in the beltl ine region of the DBNPS RPV would not significantly affect plant safety and would not have a significant adverse effect on the probability of an accident occurring. The proposed action would not result in an increased radiological hazard beyond those previously analyzed in the Final Safety Analysis Report for DBNPS.

The NRC staff’s safety evaluation will be provided in the exemption that will be issued as part of the letter to the licensee approving the exemption to the regulation, if granted.

There will be no change to radioactive effluents that effect radiation exposures to plant workers and members of the public. The proposed action does not involve a change to plant buildings or land areas on the DBNPS site. Therefore, no changes or different types of radiological impacts are expected as a result of the proposed exemption.

The proposed action does not result in changes to land use or water use, or result in changes to the quality or quantity of non-radiological effluents. No changes to the National Pollution Discharge Elimination System permit are needed. No effects on the aquatic or terrestrial habitat in the vicinity or the plant, or to threatened, endangered, or protected species under the Endangered Species Act, or impacts to essential fish habitat covered by the Magnuson–Stevens’s Act are expected. There are no impacts to the air or ambient air quality.

There are no impacts to historical and cultural resources. There would be no impact to socioeconomic resources. Therefore, no changes to or different types of non-radiological environmental impacts are expected as a result of the proposed exemption.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.